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- What does it **do**?
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- **Where** should we keep it?

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“Spent” fuel from a typical (old) reactor:

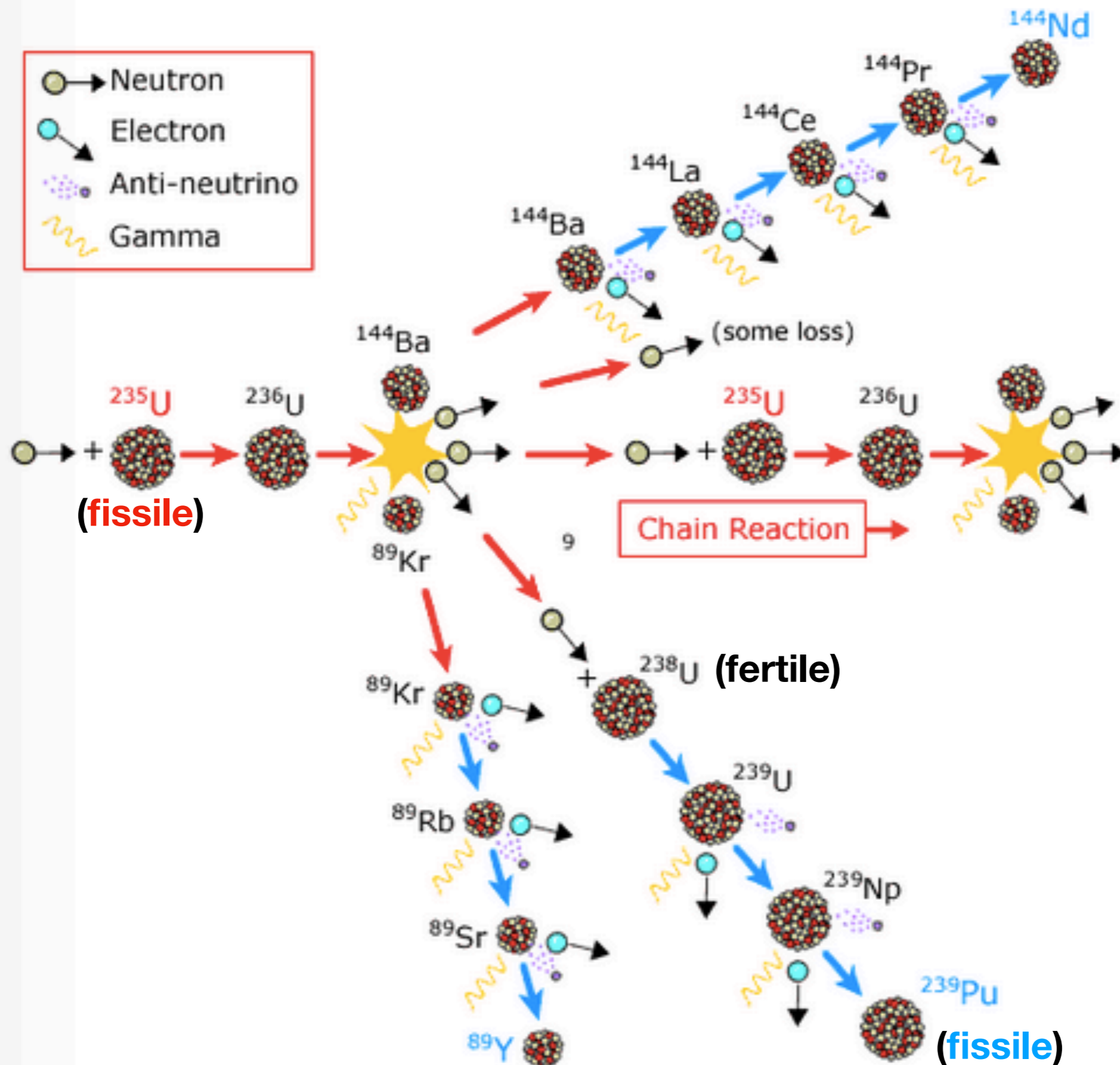
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- **Where** should we keep it?
- Is it **safe**?

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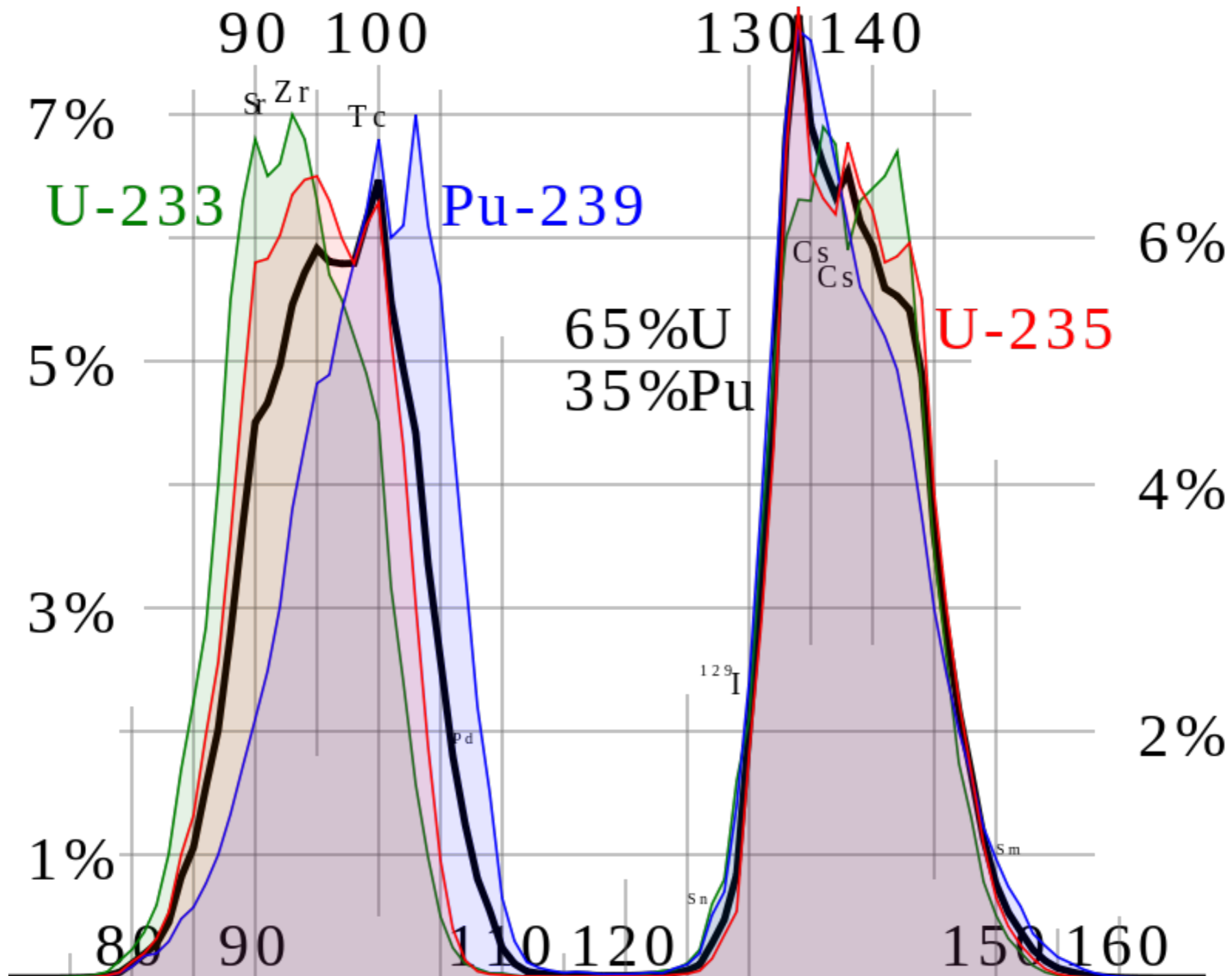
“Spent” fuel from a typical (old) reactor:

- What’s **in** it?
- What does it **do**?
- How **long** does it last?
- **Where** should we keep it?
- Is it **safe**?
- How much does it **cost** to maintain?

Transmutation tricks with Neutrons:



Fission Products

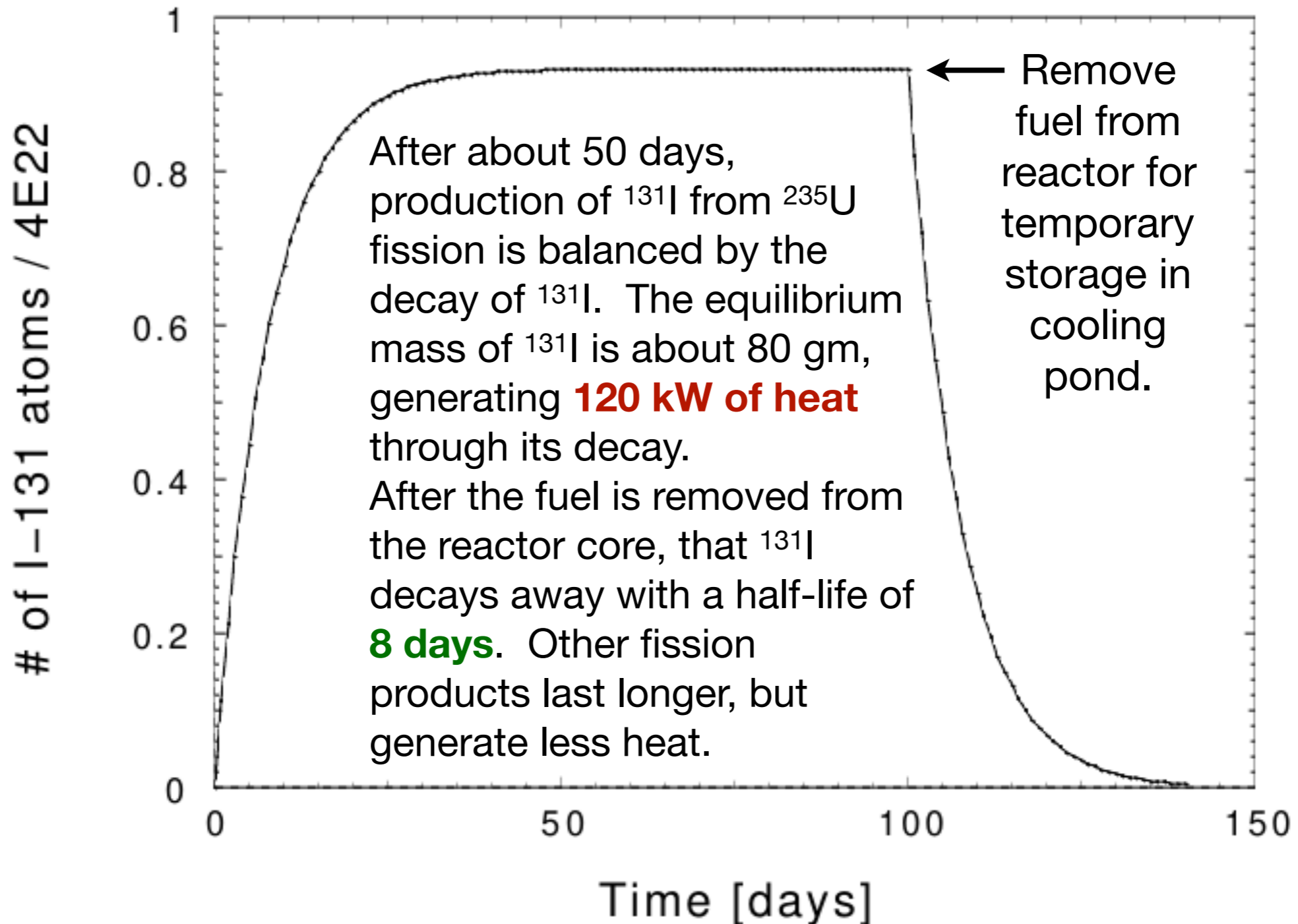


Health Concerns

Isotope	Radiation	Half-life	GI absorption	Notes
Strontium-90/yttrium-90	β	28 years	30%	
Caesium-137	β, γ	30 years	100%	
Promethium-147	β	2.6 years	0.01%	
Cerium-144	β, γ	285 days	0.01%	
Ruthenium-106/rhodium-106	β, γ	1.0 years	0.03%	
Zirconium-95	β, γ	65 days	0.01%	
Strontium-89	β	51 days	30%	
Ruthenium-103	β, γ	39.7 days	0.03%	
Niobium-95	β, γ	35 days	0.01%	
Cerium-141	β, γ	33 days	0.01%	
Barium-140/lanthanum-140	β, γ	12.8 days	5%	
Iodine-131	β, γ	8.05 days	100%	
Tritium	β	12.3 years	100%	[a]

^{131}I inventory

Buildup & Decay of I-131 in a typical reactor



Short-Term Storage



Central Interim Storage Facility (CLAB), Sweden. Image: SKB

Decay of Fission Products

Wikipedia: “The radioactivity in the fission product mixture is [initially] mostly *short-lived* isotopes such as ^{131}I and ^{140}Ba ; after about four months ^{141}Ce , $^{95}\text{Zr}/^{95}\text{Nb}$ and ^{89}Sr take the largest share, while after about two or three years the largest share is taken by $^{144}\text{Ce}/^{144}\text{Pr}$, $^{106}\text{Ru}/^{106}\text{Rh}$ and ^{147}Pm .”

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Medium-lived Daughters

Prop:	$t_{1/2}$	Yield	Q *	$\beta\gamma$ *
Unit:	(a)	(%)	(keV)	
^{155}Eu	4.76	<u>0.0803</u>	252	$\beta\gamma$
^{85}Kr	10.76	<u>0.2180</u>	687	$\beta\gamma$
$^{113\text{m}}\text{Cd}$	14.1	<u>0.0008</u>	316	β
^{90}Sr	28.9	4.505	<u>2826</u>	β
^{137}Cs	30.23	6.337	<u>1176</u>	$\beta\gamma$
$^{121\text{m}}\text{Sn}$	43.9	0.00005	390	$\beta\gamma$
^{151}Sm	88.8	<u>0.5314</u>	77	β

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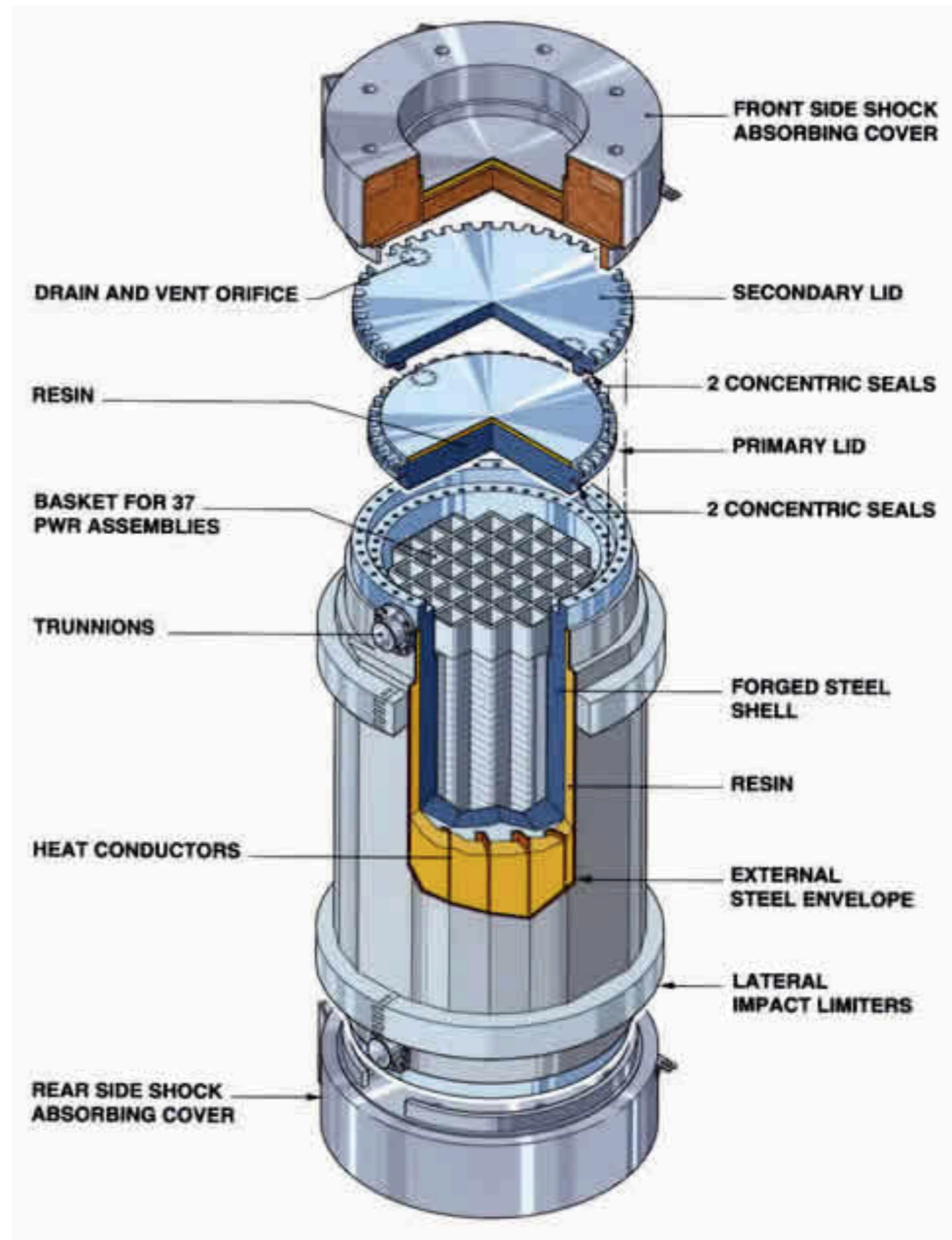
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Long-lived Daughters

Nuclide	$t_{1/2}$	Yield	Decay energy ^[a 1]	Decay mode
	(Ma)	(%) ^[a 2]	(keV)	
^{99}Tc	0.211	6.1385	294	β
^{126}Sn	0.230	0.1084	4050 ^[a 3]	$\beta\gamma$
^{79}Se	0.327	0.0447	151	β
^{93}Zr	1.53	5.4575	91	$\beta\gamma$
^{135}Cs	2.3	6.9110 ^[a 4]	269	β
^{107}Pd	6.5	1.2499	33	β
^{129}I	15.7	0.8410	194	$\beta\gamma$

Accessible Storage ("dry cask")



TN24 cask produced by Orano TN (formerly Areva TN)

Permanent Storage

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- WIPP
- Yucca Mtn
- France
- Sweden
- Finland

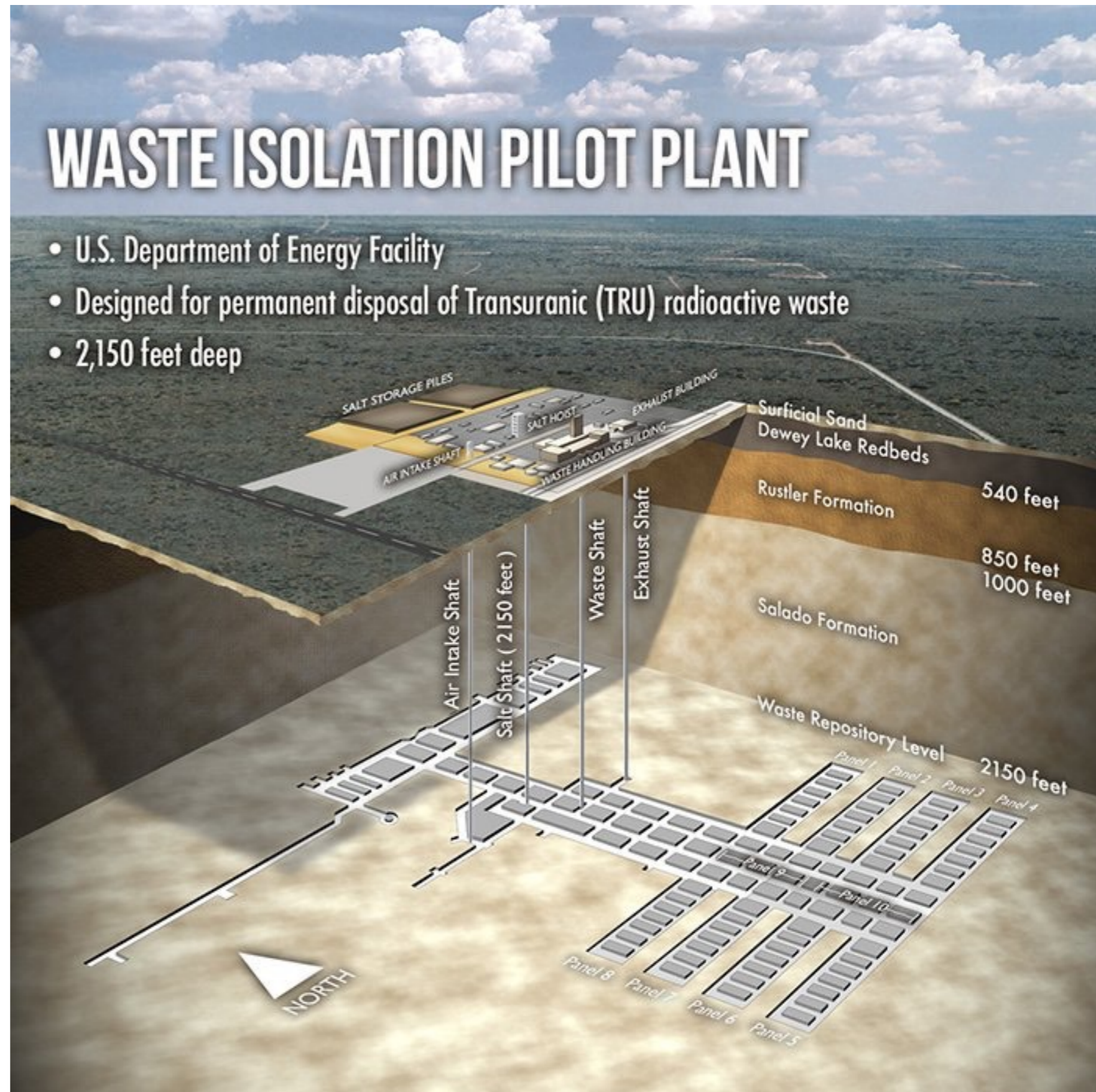
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Waste Isolation Pilot Plant (WIPP)

US DOE stores rad. waste (from nuclear weapons mfg.) 660 m underground in a salt basin near Carlsbad, NM



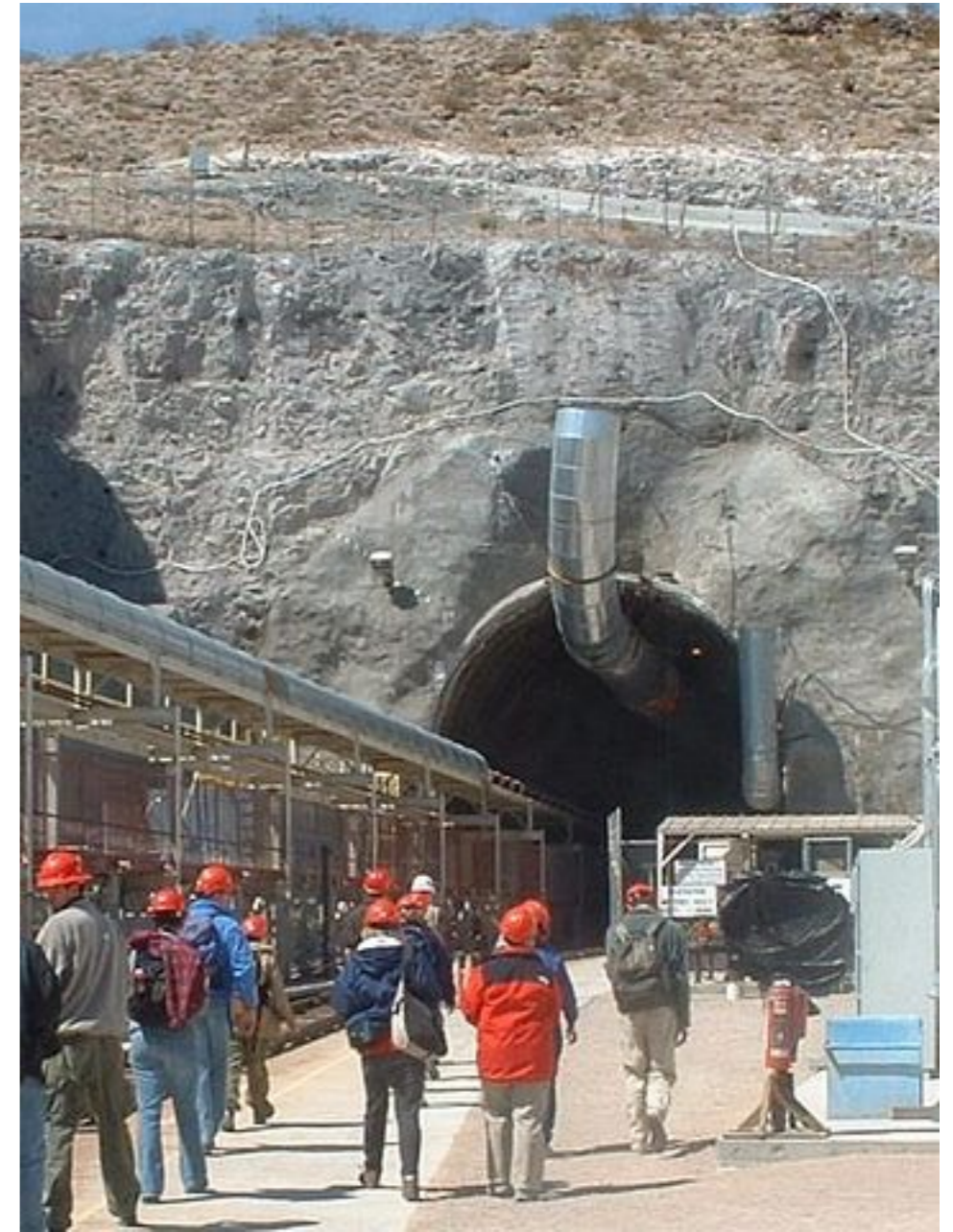
A breach in Feb 2014 released a small amount of radioactivity to the local environment. No one was harmed.



Yucca Mountain



Public Domain, <https://commons.wikimedia.org/w/index.php?curid=1026543>



By Daniel Mayer - Own work, CC BY-SA 3.0, <https://commons.wikimedia.org/w/index.php?curid=100567>

\$US 9 billion project (73% funded by tax on nuclear power) cancelled in 2009 due to opposition by Nevada residents (site is 130 km from Las Vegas).

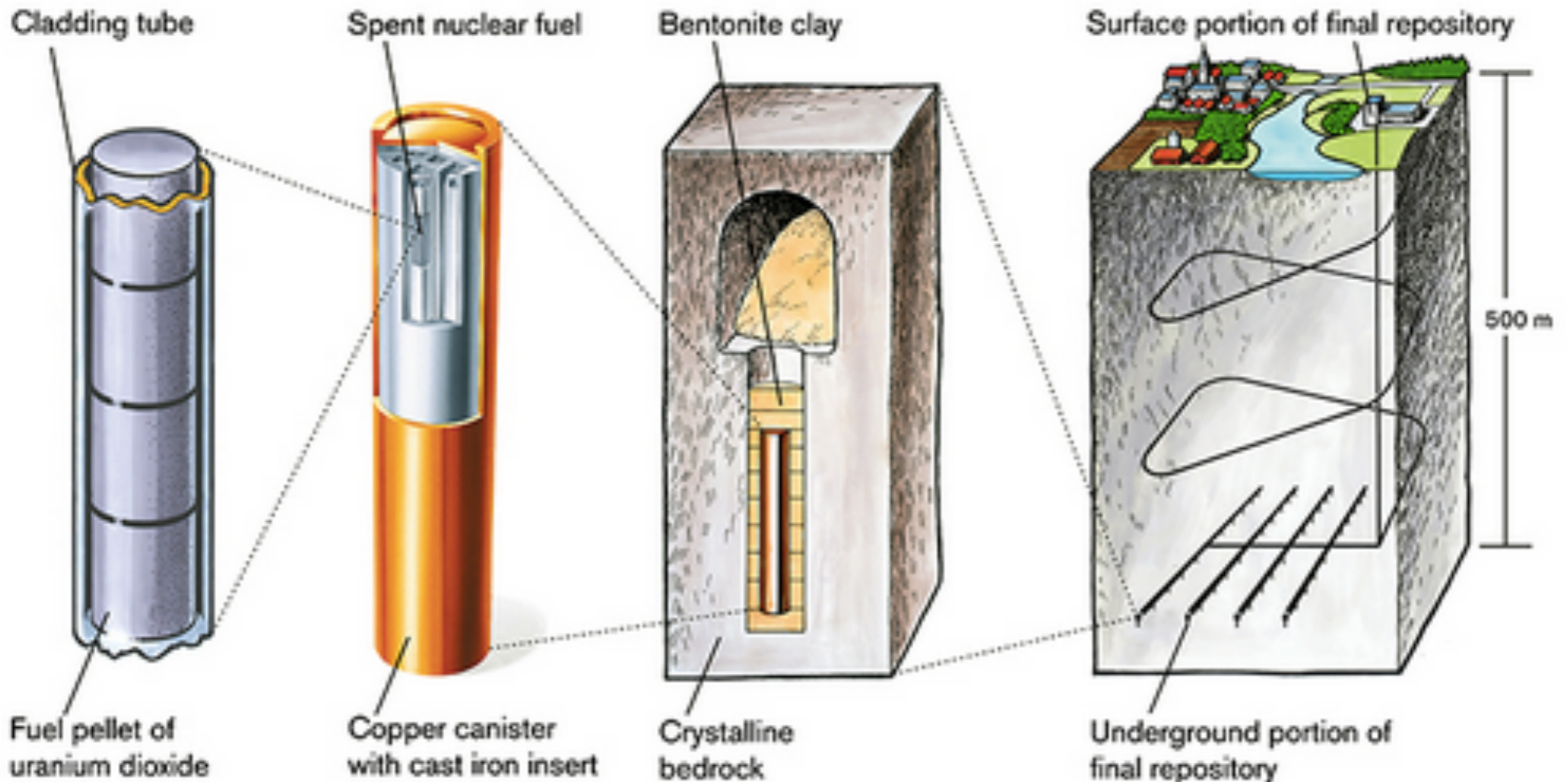
Orano La Hague (France)

Fuel Recycling facility + Short- & Long-term Storage



SKB: Forsmark, Sweden

KBS-3 method is based on three protective barriers:
copper canisters, Bentonite clay and the Swedish bedrock.



Finland: Olkiluoto Island

430 m down in bedrock

up to 50 km of tunnels



Canada: *still thinking about it.*

The Canadian Shield offers some of the world's most stable deep bedrock — but *why bury something so valuable?*

Especially when we can use it as *fuel*
in a **Molten Salt Reactor!**

MSR

Molten Salt Reactor

